CONDITIONS OF THE FLUID IN THE RHRS PIPE LINE AFTER THE LOSS AND RESTART OF THE RHRS PUMP

Dr. Ching Guey/FP&L
700 University Blvd
Juno Beach, FL 33408

Phone: 561 694 3137, Pager: 561 722 3860
e-mail: ching_guey@fpl.com

Joseph S. Miller/EDA, Inc.
2015 Woodford Rd
Vienna, VA 22182

Phone: 703 597-2459, Fax: 703 356 4149
e-mail: JoeMiller@edasolutions.com

GPM – Gallons per Minute
MOV – Motor Operated Valve
PWR – Pressurized Water Reactor
RCS – Reactor Coolant System
RHRS – Residual Heat Removal System
SDC – Shutdown Cooling

ABSTRACT

Due to a recent event at pressurized water reactor (PWR), the impact of a loss of shutdown cooling during reduced RCS inventory conditions was evaluated. The evaluation for the loss of RHRS assumed loss of shutdown cooling initially and then the RHRS pumps were restored. This analysis is based upon a decay heat rate corresponding to 2.66 days following reactor shutdown with an initial liquid level at the elevation of the reactor flange. When the RHRS pump was restored, the calculation revealed that a void fraction of 10 -50 % could exist in the hot leg pipe where the RHRS pumps take suction.

The study presented in this paper was conducted to determine the maximum voids permissible in the hot leg piping and still ensure that the water entering the RHRS pump did not have steam voids. RELAP5 was used to perform this evaluation. RELAP5/M3.3 is a computer code used for the thermal-hydraulic analysis of transients and small-break accidents and transients in light-water nuclear power plants. It uses a one-dimensional, two-fluid model, consisting of steam and water, with the possibility of the vapor phase containing a non-condensable component. A model of the piping was developed and the conditions presented above were analyzed and the fluid conditions were reviewed.

The results of the RELAP5 analyses indicated that for voids 30 % or less in the hot leg, the RHRS pump would have adequate suction flow to remain in a stable operational condition.

1. INTRODUCTION

Due to a recent outage event at a PWR, a loss of shutdown cooling for 8 minutes occurred during reduced RCS inventory conditions. The head was still on the vessel and water level was at the flange, or approximately 7 ft from Mid-loop of the hot leg. A calculation was performed to evaluate the system response. The calculation showed the possibility of voids in the hot leg piping where the RHRS pump takes suction for shutdown cooling. To determine the safety significance of the event, a concern was raised that the RHRS pumps may not operate with voids in the hot leg piping. To provide evidence that the RHRS pumps will operate with some voiding in the hot leg piping, a RELAP5 model of the RHR piping was constructed and used to perform simulations of hot leg voiding scenarios.

2. DISCUSSION

A RELAP5/M3.3 model was constructed to simulate the fluid in the RHRS line from the Hot Leg to the RHRS pump. RELAP5/M3.3 (Ref. 1) is a computer code used for the thermal-hydraulic analysis of transients and small-break accidents and transients in light-water nuclear power plants. RELAP5 uses a one-dimensional, two-fluid model, consisting of steam and water, with the possibility of the vapor phase containing a non-condensable component. The RELAP5 program has been used to solve similar problems to the RHRS flow problem (Refs. 2 & 3) and has been validated for calculating fluid flow conditions and resulting forcing functions (Ref. 4).

The RELAP5 model was constructed for the RHRS system and is shown in Figure 1. The model begins from the hot leg (RELAP5 component 104), which is a 30-inch pipe leaving the reactor vessel. The 14-inch Residual Heat Removal (RHR) connects to the hot leg through junction 401 and traverses to RHRS pump (RELAP5 Component 201) as shown in the diagram in Figure 1.

The model consists of two pumps, i.e., RHRS pumps A and B. For these simulations, pump B (RELAP5 Component 202) was not started. The pumps rated head is 210 feet at a rated flow of 3500 gpm. Two motor operated valves (MOVs), were also modeled in series. Loss coefficients and sizes were taken from References 5, 6 & 7. At time equal zero, the RHRS 14" piping is full of 120° F water. The 14" pipe running from the Hot Leg to pump A is about 190 feet in length.

3. RESULTS

Four cases were analyzed with RELAP5. The first case simulated saturated conditions at 55 psia in the hot leg pipe with the pump A running at a rated flow of 3500 gpm at rated density. The second case was to simulate slightly less than 10% voids in the Hot Leg piping at 55 psia. The third case was to simulate 30% voids in the Hot Leg piping at 55 psia and the fourth case was to simulate greater than 40% in the Hot Leg piping at 55 psia.

Figure 3 Densities for Saturated Liquid in Hot Leg

Figure 4 Voids in the 14” RHRS Inlet for the 4 Cases Simulated

Figure 4 shows the voids in the 14” RHRS pipe inlet for the 4 cases. Case 1 assumes saturated liquid in the hot leg, Case 2 assumes slightly less than 10% voids in the hot leg, Case 3 assumes 30 % voids in the hot leg and Case 4 assumes greater than 40% voids in the hot leg.

Figure 5 shows the density at the RHRS pump inlet for each of the cases simulated. For Cases 1, 2 and 3 (i.e., saturated liquid in the hot leg, less than 10% voids in the hot let and 30% voids in the hot leg, respectively), the density at the RHRS pump inlet was calculated as 58 lbm/ft³. For Case 4, the greater than 40% voids in the hot leg case, the density was calculated to be .25 lbm/ft³ and the pump failed to adequately pump fluid.

Figure 5 Density in the 13” RHRS Pump Inlet for the 4 Cases Simulated

Figure 6 Voids in the 14” RHRS Pump Inlet for the 4 Cases Simulated

Figure 6 shows the steam void at the RHRS pump inlet for each of the cases simulated. For Cases 1, 2 and 3 (i.e., saturated liquid in the Hot Leg, less than 10% voids in hot let and 30% voids in hot leg, respectively), the void at the RHRS pump inlet was calculated as 0.0. For Case 4, the greater than 40% voids in the hot leg case, the void was calculated to be 1.0 and the pump failed to pump fluid.

Table 1 presents the results of the 4 cases simulated. Cases 1, 2 and 3 that represented up to 30% voiding in the hot leg, indicated that no voiding occurred in the pump inlet. Case 4, which represented the simulated case with greater than 40% voids in the hot leg, showed that the pump inlet would be completely voided with no flow through the pump.

Table 1 Summary of Results for all Cases

RELAP5 Case	Description	RELAP5 File	Voids in Hot Leg	Voids at Pump 3A Inlet
1	Saturated Liquid in Hot Leg	RHRpg.i	None	None
2	Less Than 10% Voids in Hot Leg	RHRpd.i	Less Than 10% Voids	None
3	30% Voids in Hot Leg	RHRpe.i	30% Voids	None
4	Greater Than 40% Voids in Hot Leg	RHRpf.i	Greater Than 40% Voids	100% Voiding

4. CONCLUSIONS

Four cases were simulated with RELAP5. Case 1 simulated saturated conditions at 55 psia in the hot leg pipe. Case 2 simulated slightly less than 10 voids in the hot leg piping at 55 psia. Case 3 simulated 30% voids in the hot leg piping at 55 psia and Case 4 simulated greater than 40% in the hot leg piping at 55 psia. For all cases, the A pump was running at a rated flow of 3500 gpm at rated density.

The results of the RELAP5 analyses indicated that for voids 30 % or less in the hot leg, the RHRS pump would have adequate suction density and flow to remain in a stable operational condition.

REFERENCES

- 1) NUREG/CR-5535/Rev1, RELAP5/MOD3.3 VOLUMES 1 through 8, Prepared by Information Systems Laboratories, Inc., Idaho Falls, ID for Division of Systems Research, NRC, Washington, DC 20555. December 2001.
- 2) "Solving Reactor Water Clean-up System Anomalies Using RELAP5MOD3.3", Joseph S. Miller and Chris Brennan, 11th International Conference on Nuclear Engineering, ICONE11, Tokyo, JAPAN, ICONE11-36149, April 20-23, 2003.
- 3) "Calculation of Forces on Reactor Containment Fan Cooler Piping", Joseph S. Miller and Kevin Ramsden, 12th International Conference on Nuclear Engineering, ICONE12, Arlington, Virginia, ICONE12-49214, April 25-29, 2004.
- 4) "Validation of a Thermal Hydraulic Computer Code to Perform Two-Phase Multi-Component Force Calculations for Structural Evaluations", Joseph S. Miller and Kevin Ramsden, 13th International Conference on Nuclear Engineering, ICONE13, Beijing, China, ICONE13-50297, May 16-21, 2005.
- 5) I.E. Idel' Chik, "Handbook of Hydraulic Resistance, Coefficients of Local Resistance and of Friction", AEC-TR-6630, (1966).
- 6) "Flow of Fluids Through Valves, Fittings and Pipe", Technical Paper No. 410, CRANE, Twenty Fifth Printing 1991.
- 7) NAVCO PIPING DATALOG, National Valve and Manufacturing Company, Edition No. 10 (Rev. June 1, 1974) 1966.